November 19, 1984 NRC/THI-84-081

MEMORANDUM FOR:	Harold R. Denton, Director Office of Nuclear Reactor Regulation
	Bernard J. Snyder, Program Director THI Program Office
FROM:	William D. Travers, Deputy Program Director THI Program Office

SUBJECT: NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT FOR NOVEMBER 10, 1984 - NOVEMBER 17, 1984

A remotely controlled robot survey vehicle was used to perform a video inspection of the 282 ft. elevation of the reactor building. This is the first inspection of this area since the 1979 accident.

Data from effluent and environmental monitoring systems indicated no plant release in excess of regulatory limits. Waste processing continued on a routine basis. Plant parameters have shown no significant changes. Other site activities this period included: robot entry, plenum assembly inspection, and continued fuel pool "A" refurbishment.

Significant items covered in the enclosure are:

- -- Reactor Building Activities
- -- Auxiliary and Fuel Handling Building Activities
- -- Waste Water Management Activities
- -- EPA Environmental Sampling Results

Summary sheets included in this report are:

- -- Liquid Effluent and Environmental Data
- -- Waste Water Processing Data
- -- Plant Status Data

ORIGINAL SIGNED BY:

William D. Travers Deputy Program Director TMI Program Office

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Enclosure: As stated

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NRC FORM 315 (10 80) NRCM 0240

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ENCLOSURE

REACTOR BUILDING ACTIVITIES:

A robotic vehicle is being used to inspect the basement of the reactor building. After two inspections, each several hours in duration, the robot has inspected the southeast half of the basement. The inspections were limited to the areas outside the "D-rings." The video tapes produced by the inspections are being reviewed and an expanded program for the robotic vehicles is being evaluated. Dose rates measured on the 282 ft. elevation during the inspection confirmed previous measurements that the basement is essentially inaccessible to humans. General area radiation fields range from 10 to 70 R/hr. One location near the elevator enclosure was measured to be in excess of 1,100 R/hr.

Other work in the reactor building is proceeding on schedule. Procedures are being finalized to commence the plenum jacking operations in early December. The plenum is scheduled to be removed from the vessel in May 1985 and defueling should commence during the summer of 1985.

AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES:

The "A" fuel pool refurbishment has continued this week. Decontamination of the stainless steel liner is approximately 20% completed.

The makeup and purification resin elution process has been on hold since October 22 due to a valve failure. Maintenance work on the valve has been substantially completed.

WASTE MANAGEMENT ACTIVITIES:

The submerged demineralizer system (SDS) completed processing batch 107 on November 7, 1984. Processing parameters for SDS batches 102 to 107 are included in Appendix 2.

The EPICOR II system completed processing batch 230 on November 7, 1984. Processing parameters of EPICOR II batches 221 to 230 are shown in Appendix 2.

The following is a summary of the processing history of both the SDS and EPICOR II systems.

SDS PROCESSING SUMMARY (July 1981 through November 15, 1984)

Water Source	Approximate Gallons		
Reactor Building Sump Reactor Building Sump (Decon) Reactor Cóolant System	651,000 443,000 797,000		
Miscellaneous Decon Water	678,000		
Total	2,569,000		

EPICOR II PROCESSING SUMMARY (November 1979 through May 1984)

<u>Water Source</u>	Approximate Gallons
Pre-SDS Auxiliary Building Sump	565,000
Reactor Building Sump	649,000
Reactor Building Sump (Decon)	400,000
Miscellaneous Sources	621,000
Total	2,235,000

EPA ENVIRONMENTAL SAMPLING RESULTS:

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Included in Appendix 1 are results from EPA's environmental monitoring program in the area surrounding Three Mile Island. Periods covered include August 17 through October 12, 1984 for noble gas monitoring, and September 1 through October 31, 1984 for gamma radiation and airborne particulate radioactivity.

APPENDIX 1

LIQUID EFFLUENT AND ENVIRONMENTAL DATA

GPU Nuclear

Based on sampling and monitoring, liquid effluents from the TMI site released to the Susquehanna River were determined to be within regulatory limits and in accordance with NRC requirements and the City of Lancaster Agreement.

During the period November 9, 1984 through November 15, 1984, liquid effluents contained no detectable radioactivity at the discharge point. Individual effluent sources originating within Unit 2 contained minute amounts of radioactivity. Calculations indicate that less than 8.6 E-7 (0.00000086) of a curie of Cs-137, less than 1.5 E-6 (0.0000015) of a curie of gross beta activity and less than 7.8 E-6 (0.0000078) of a curie of tritium were discharged.

Environmental Protection Agency

Lancaster Water Samples: 7 samples

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Period Covered:	October 27 - November 3, 1984
Results:	Gamma Scan Negative for reactor related radioactivity
II Water Samples:	7 samples
Period Covered:	October 28 - November 3, 1984
Results:	Gamma Scan Negative for reactor related radioactivity

NRC Environmental Data

The NRC operated continuous outdoor air sampler at the TMI site did not detect any reactor related radioactivity. The air sampler parameters are listed below. The analysis results were less than the lower limit of detectability of the analytical instruments: 7.7 E-14 uCi/cc for I-131 and 7.7 E-14 uCi/cc for Cs-137.

Sample	Period				Volume
HP-445	November 11	- November	15, 1	984	455.8m ³

EPA Environmental Data

-- The EPA measures Kr-85 concentrations in air at several environmental monitoring stations and reported the following results:

Location	August 17-31, 1984 (pCi/m ³)	August 31 - September 14, 1984 (pCi/m ³)
Goldsboro Middletown	25 26	26 27 Samplar Malfunction
TMI Observation Cente	r 25	27
<u>Location</u>	September 14-28, 1984 (pCi/m ³)	<u>September 28 - October 12, 1984</u> (pCi/m ³)
Goldsboro Middletown Yorkhaven TMI Observation Cente	25 25 25 r 22	26 25 26 25

-- The EPA gamma radiation detection system continuously monitors for increases above naturally occurring radioactivity and residual fallout radioactivity at 13 stations in the TMI area. During this period the EPA has attributed the measurements to naturally occurring radioactivity and/or residual fallout radioactivity.

Period Covered: September 1-30, 1984

Location	Direction	Distance	Average	Integrated Dose
	(degrees)	(miles) (nillirem/hr)	(millirem)
Harrisburg International				
Airport, Middletown	325	3.5	.008	6.0
Londonderry Township Bldg	040	2.6	.007	5.2
Newville	100	3.0	.010	7.1
Falmouth	130	2.9	.010	7.3
Falmouth	150	3.0	.008	5.8
York Haven	180	3.0	.009	6.3
Woodside	205	2.5	.007	4.9
Goldsboro	270	1.5	.011	7.7
Plainfield	305	2.7	.008	5.3
Royalton	068	3.5	.010	6.8
TMI Observation Center	095	0.5	.008	5.7
EPA TMI Field Station,				
Middletown	356	2.8	.006	4.5
Newberrytown	136	3.0	.009	6.3
Yocumtown	275	4.0	.007	5.2

Period Covered: October 1-31, 1984

Location	Direction (degrees)	Distance (miles) (n	<u>Average</u> nillirem/hr)	Integrated Dose (millirem)
Harrisburg International				
Airport, Middletown	325	3.5	.008	6.3
Londonderry Township Bldg	040	2.6	.007	5.2
Newville	100	3.0	.010	6.9
Falmouth	130	2.9	.010	7.3
Falmouth	150	3.0	.008	5.7
York Haven	180	3.0	.009	6.3
Woodside	205	2.5	.007	5.0
Goldsboro	270	1.5	.011	7.8
Plainfield	305	2.7	.008	5.4
Royalton	068	3.5	.009	6.6
TMI Observation Center	095	0.5	.008	5.6
EPA TMI Field Station,				
Middletown	356	2.8	.006	4.5
Newberrytown	136	3.0	.009	6.3
Yocumtown	275	4.0	.007	5.1

-- EPA results of airborne particulate samples collected at the same locations as the gamma radioactivity monitors (above) during the period September 1 - October 31, 1984 were all less than 0.2 picocuries per cubic meter of air, the minimum detectable concentrations for EPA's analytical instruments.

APPENDIX 2

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WASTE WATER PROCESSING DATA

Submerged Demineralizer System (SDS)

Batch	Source	(gallons)	Processing-Dates
102	"C" Reactor Coolant Bleed Tank	5,107	August 16-17, 1984
103	Reactor Coolant System (IIF)	56,539	August 17-20, 1984
104	Neutralizer Tanks	6,382	October 6-7, 1984
105	Neutralizer Tanks	10,005	October 19-20, 1984
106	Neutralizer Tanks	12,981	October 27-28, 1984
107	Reactor Building Sump	41,275	November 1-7, 1984

SDS Performance Parameters

<u>Batch</u>	Radionuclide	Average Influent (uc/ml)	Average Effluent (uc/ml)	Percent <u>Removed</u> (%)
102	Cesium 137	2.6 E-2	3.7 E-5	99.86
	Strontium 90	1.3 E-1	2.9 E-3	97.77
103	Cesium 137	5.9 E-1	5.1 E-5	99.99
	Strontium 90	3.9 EO	5.9 E-2	98.49
104	Cesium 137	4.2 EO	4.8 E-3	99.99
	Strontium 90	1.3 EO	5.3 E-3	99.59
105	Cesium 137	2.6 E+1	3.1 E-2	99.88
	Strontium 90	5.6 E-1	3.1 E-2	94.41
106	Cesium 137	6.3 EO	2.8 E-2	99.56
	Strontium 90	3.5 E-1	2.6 E-2	92.48 ·
107	Cesium 137	5.0 E0	8.0 E-4	99.98
	Strontium 90	1.7 E0	1.6 E-3	99.91

EPICOR II

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Batch	Source	(galions)	Processing-Dates
221	"A" Reactor Coolant Bleed Tank	17,000	August 10-11, 1984
222	EPICOR Building Sump	3,084	August 11, 1984
223	"A" Monitor Tank	10,535	August 22, 1984
224 .	"B" Monitor Tank	10,089	August 22-23, 1984
225	"A" Monitor Tank	10,821	October 20-21, 1984
226	"B" Monitor Tank	10,855	October 28, 1984
227	"A" Monitor Tank	8,983	November 1, 1984
228	"B" Monitor Tank	10,916	November 3-4, 1984
229	"A" Monitor Tank	10,692	November 5, 1984
230	"B" Monitor Tank	10,915	November 6-7, 1984

EPICOR Performance Parameters

Batch	Radionuclide	Average Influent (uc/ml)	Average Effluent (uc/ml)	Percent Removed (%)
221	Cesium 137	5.1 E-3	1.2 E-7	99.99
	Antimony 125	2.6 E-2	4.9 E-7	99.99
	Strontium 90	2.8 E-2	6.9 E-6	99.98
222	Cesium 137 Antimony 125 Strontium 90	3.2 E-4 2.7 E-4	2.0 E-7 3.4 E-7 6.6 E-7	99.94 99.76
223	Cesium 137	1.2 E-4	1.7 E-7	99.86
	Antimony 125	5.1 E-3	3.1 E-7	99.99
	Strontium 90	8.1 E-3	5.8 E-6	99.93
224	Cesium 137	9.6 E-5	1.9 E-7	99.81
	Antimony 125	5.4 E-3	3.2 E-7	99.99
	Strontium 90	1.9 E-2	9.5 E-6	99.95
225	Cesium 137	1.3 E-2	4.5 E-7	99.99
	Antimony 125	3.1 E-3	6.2 E-7	99.98
	Strontium 90	1.2 E-2	3.0 E-5	99.75

EPICOR Performance Parameters (continued)

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Batch	Radionuclide	Average Influent (uc/ml)	Average <u>Effluent</u> (uc/ml)	Percent Removed (%)
226	Cesium 137	2.8 E-2	1.5 E-6	99.99
	Antimony 125	3.4 E-3	6.1 E-7	99.98
	Strontium 90	2.2 E-2	1.1 E-5	99.95
227	Cesium 137	2.3 E-2	1.9 E-7	99.99
	Antimony 125	1.7 E-3	2.9 E-7	99.98
	Strontium 90	2.2 E-2	8.8 E-6	99.96
228	Cesium 137	3.6 E-4	1.3 E-7	99.96
	Antimony 125	9.4 E-3	3.6 E-7	99.99
	Strontium 90	3.6 E-3	1.7 E-5	99.53
229	Cesium 137	9.9 E-4	3.1 E-7	99.97
	Antimony 125	8.4 E-3	6.5 E-7	95.99
	Strontium 90	1.1 E-3	1.3 E-5	99.80
230	Cesium 137	3.4 E-4	4.1 E-7	99.88
	Antimony 125	9.5 E-3	5.3 E-7	99.99
	Strontium 90	7.9 E-4	1.1 E-5	99.61

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APPENDIX 3

PLANT STATUS

Reactor Vessel Configuration: Reactor vessel open with modified internals indexing fixture installed

Core Cooling Mode: Heat transfer from the reactor coolant system (RCS) to reactor building ambient

Available Core Cooling/Makeup Sources: Standby pressure control (SPC) system Reactor coolant bleed tank (RCBT) water transfer system Mini decay heat removal (MDHR) system

Major Parameters as of 6:00 AM, November 16, 1984 (approximate values):

Reactor Coolant System:

Loop Temperatures:

	A	В
Cold Leg (1)	65°F	68°F
(2)	67°F	69°F

Reactor Core:

Average Incore Thermocouples:* 94°F Maximum Incore Thermocouple:* 105°F Decay Heat: 15 kilowatts

Reactor Building: Temperature: 63°F Pressure: -0.03 psig

Airborne Radionuclide Concentrations:

Tritium: 2.5 E-9 uCi/cc (sample 11/13/84) Particulates: 3.6 E-9 uCi/cc (sample 11/15/84) predominately Cs-137

*Uncertainties exist as to the exact location and accuracy of these readings.